

# **A COMPARISON OF COMPUTATIONAL APPROACHES FOR THE DETECTION OF GAMMA-EMITTING RADIONUCLIDES IN FOODS**

**Clarence J. Rolle**

**Office of Regulatory Affairs (ORA)**

**Winchester Engineering Analytical Center (WEAC)**

**Food and Drug Administration (FDA)**

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# Food Emergency Response Network



- Interest :** Quick and sound food safety decision-making in the event of nuclear or radiological emergency
- Objectives:** Develop rapid and high-throughput screening methods in support of large-scale emergency response and consequence management
- Increase radioanalytical capability and sample-surge capacity with greater geographic coverage and operational flexibility
- Approach:** Communicate and collaborate with state laboratories in method development, validation, and standardization
- Provide resources, trainings, and proficiency tests
- Deliverable:** Full range of radioanalytical capability for detecting all radionuclides concerning food safety
- Ability of providing sufficient & reliable analytical results to enable prompt decision-making on food safety



## Radiation in Foods

- Long-term consumption of food contaminated with high-level radioactivity increases health risk from exposure to radiation.
- Iodine 131, a gamma & beta-emitting radionuclide, can rapidly incorporated into milk and accumulate in thyroid if ingested. It increases risk of thyroid cancer, particularly in young children who consume large amount of milk products.
- Cesium 137, another gamma & beta-emitting radionuclide, behaves like potassium and can appear in meat. Cesium-137 has a half-life of 30 years, which can cause long-term food contamination. Prolonged exposure to Cs-137 can lead to a wide range of cancers.
- Strontium 90, a beta-emitting radionuclide, has chemical similarity to calcium, which can accumulate in bone. A direct linkage between strontium 90 accumulation and bone cancer and leukemia has been found.

# Radionuclides of Concern



Radionuclide	Principal Emission	Radionuclide	Principal Emission	Radionuclide	Principal Emission
<sup>241</sup> Am	α	<sup>137</sup> Cs	γ	<sup>89</sup> Sr	β
<sup>238</sup> Pu	α	<sup>60</sup> Co	γ	<sup>90</sup> Sr/ <sup>90</sup> Y	β
<sup>239</sup> Pu	α	<sup>153</sup> Gd	γ	<sup>147</sup> Pm	β
<sup>240</sup> Pu	α	<sup>192</sup> Ir	γ	<sup>227</sup> Ac	β
<sup>210</sup> Po	α	<sup>75</sup> Se	γ	<sup>3</sup> H	β
<sup>242</sup> Cm	α	<sup>170</sup> Tm	γ	<sup>32</sup> P	β
<sup>243</sup> Cm	α	<sup>169</sup> Yb	γ	<sup>241</sup> Pu	β
<sup>244</sup> Cm	α	<sup>141</sup> Ce	γ	<sup>228</sup> Ra	β
<sup>252</sup> Cf	α	<sup>144</sup> Ce	γ	<sup>63</sup> Ni	β
<sup>226</sup> Ra	α	<sup>57</sup> Co	γ	<sup>99</sup> Tc	β
<sup>237</sup> Np	α	<sup>134</sup> Cs	γ	<sup>204</sup> Tl	β
<sup>228</sup> Th	α	<sup>125</sup> I	γ	<sup>14</sup> C	β
<sup>230</sup> Th	α	<sup>129</sup> I	γ	<sup>85</sup> Kr	β
<sup>232</sup> Th	α	<sup>131</sup> I	γ	<sup>68</sup> Ge	EC
<sup>234</sup> U	α	<sup>99</sup> Mo	γ	<sup>55</sup> Fe	EC
<sup>235</sup> U	α	<sup>103</sup> Pd	γ		
<sup>238</sup> U	α	<sup>103</sup> Ru	γ		
		<sup>106</sup> Ru	γ		
		<sup>198</sup> Au	γ		
		<sup>109</sup> Cd	γ		
		<sup>99m</sup> Tc	γ		
		<sup>140</sup> Ba	γ		
		<sup>140</sup> La	γ		
		<sup>65</sup> Zn	γ		
		<sup>85</sup> Sr	γ		
		<sup>7</sup> Be	γ		

## Sources of Radionuclides:

- Natural Origin
- Weapon Tests
- Nuclear Power
- Mining Industries
- Space Explorations
- Nuclear Medicine
- Waste Disposals
- Terrorist Activities

# Detector System & Sample Preparation



**Front of Detector**



**Back of Detector**



[www.fda.gov](http://www.fda.gov)  
**Sample & 400-mL Container**



**Loading Test Sample**

# CURRENT APPROACH



## Gamma Spectrometry

Can be used to identify and quantify radionuclides that are gamma emitters

## Advantages

Non-destructive technique

Multi-radionuclide analysis

For Identification – Energy calibration

Standard sources containing multiple radionuclides with known photon energies

For Quantification – Efficiency calibration

Requires the equivalent volume geometry between the calibration standard and the test samples

# Computational Approach

Software packages have been developed and can be applied to calculate counting efficiency without using calibration standards

## Lower Cost than Source-based Calibration

Purchasing Sources – For calibrations

Disposal of Sources – Radioactive Waste

Replacing Sources – Radioactive Decay

## Time

Less Labor Intensive - Easier to match the sample for calibration

## Flexibility

Limited amount of sample

Unusual material composition





# Software Packages For Efficiency Calculations

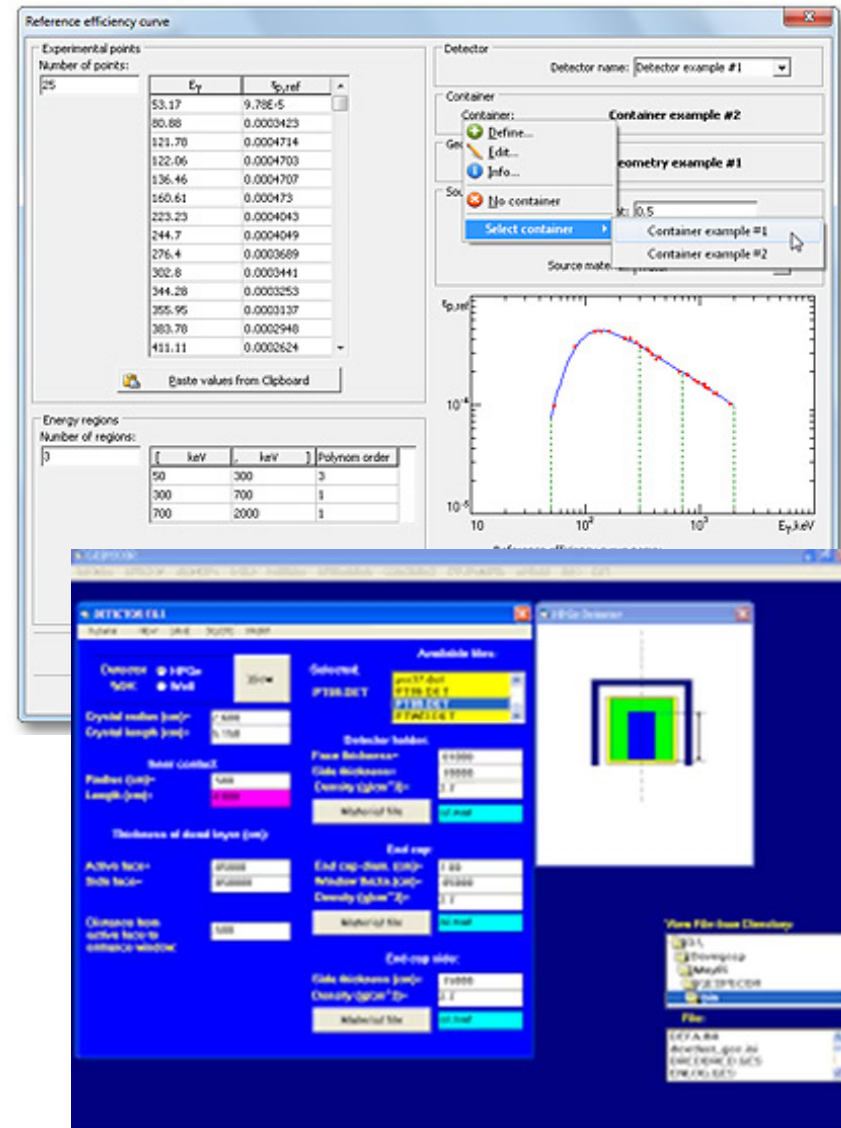
**ANGLE** combines advantages of both absolute (methods – minimizing potential for systematic errors in the former and reducing practical limitations of the latter.

## GESPECOR

The software is very flexible and allows the user to change many variables, including the detector, shield and geometry all from a friendly user interface.

## LABSOCS/ISOCS

The difference between LabSOCS and other available mathematical calibration packages is that the detector used to perform activity measurements is initially characterized by the manufacturer.





# Computational Setup

Computational approach requires descriptive inputs with regard to:

- Detector Characteristics
- Sample Properties
- Shielding Material
- Counting Geometry Arrangement

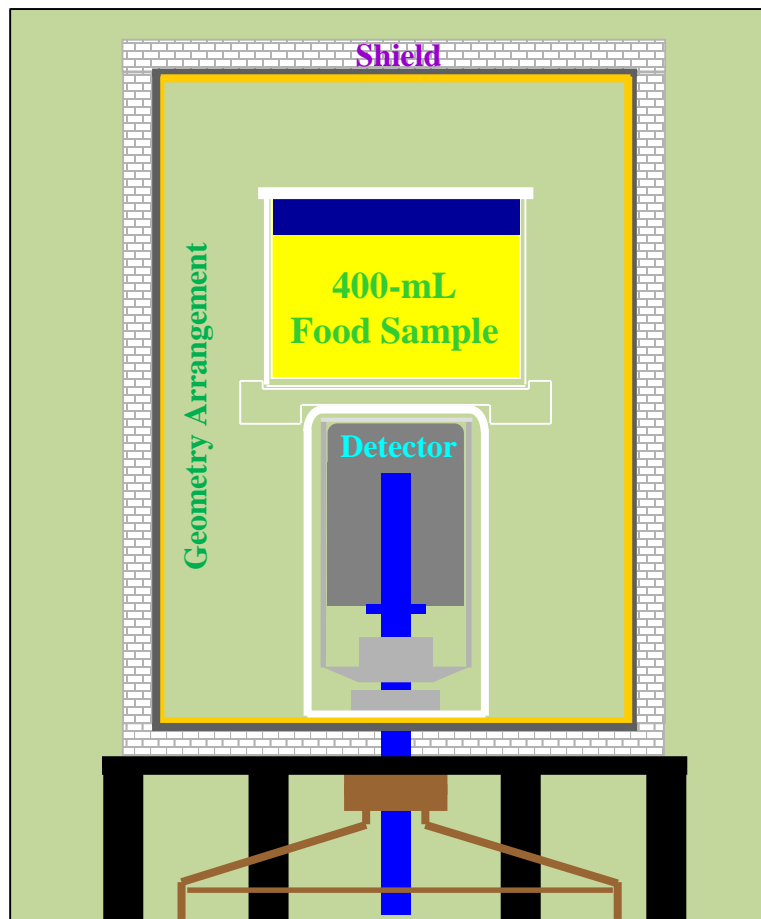
## List of Input Variables

Crystal Radius  
Crystal Length  
Crystal Inner Contact Radius  
Crystal Inner Contact Length  
Crystal Holder Face Thickness  
Crystal Holder Side Thickness  
Crystal Holder Material Density  
Face Dead Layer Thickness  
Side Dead Layer Thickness  
End Cap Diameter  
End Cap Window Thickness  
End Cap Side Thickness  
End Cap Material Density

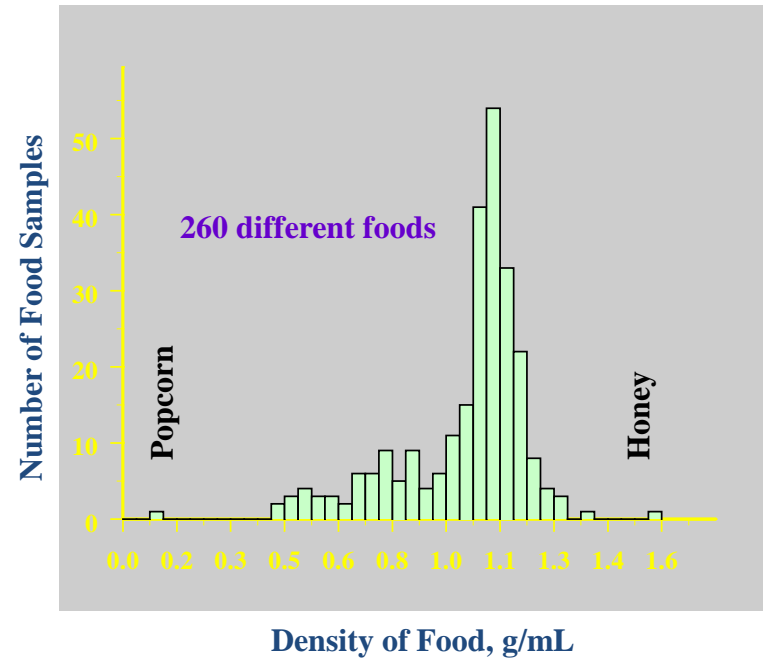
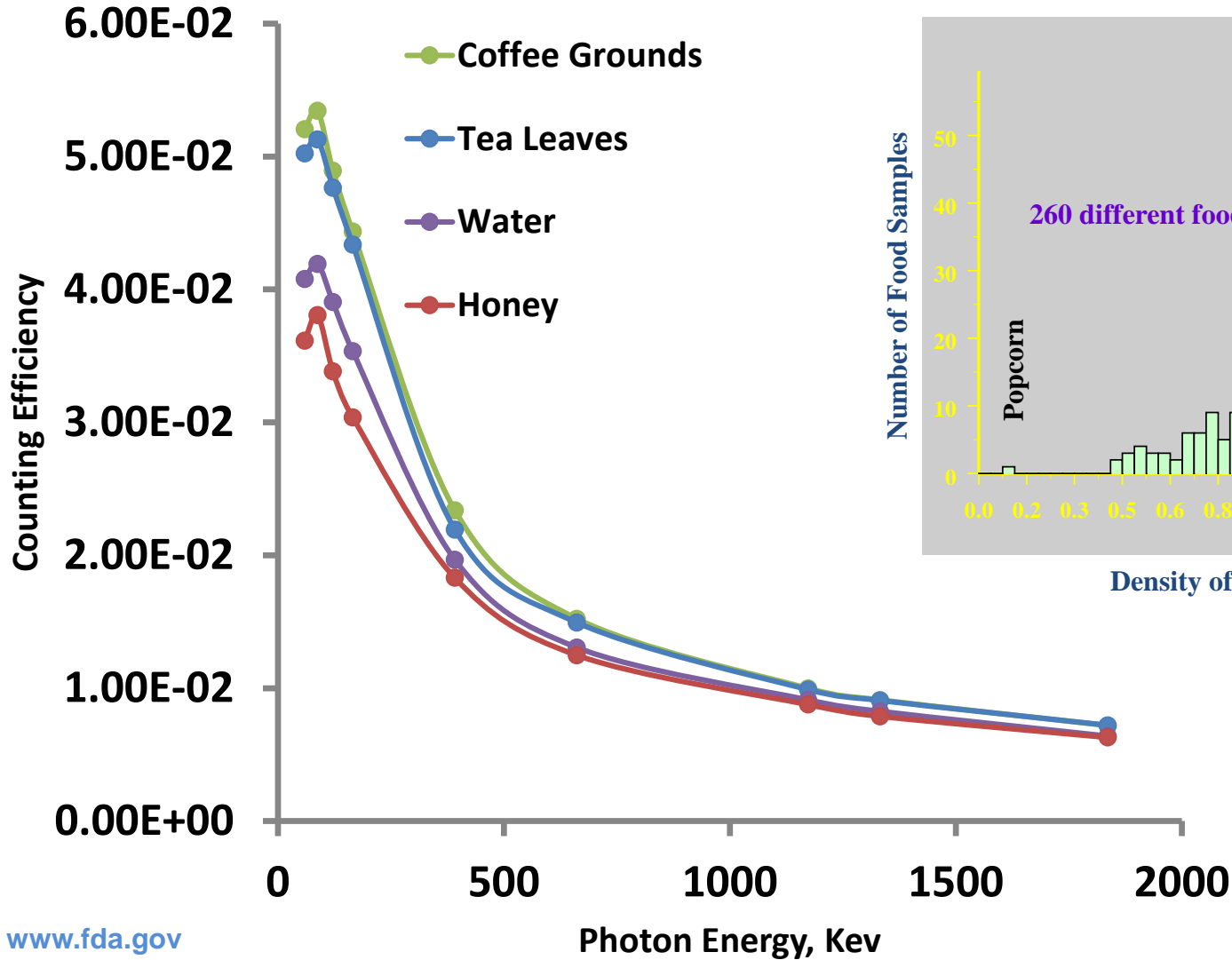
Container Radius  
Container Height  
Container Wall Thickness  
Container Material Density  
Sample Composition  
Sample Density

Shield Inner Radius  
Shield Inner Height  
Shield Outer Radius  
Shield Outer Height  
Shielding Material Density

End Cap-to-Inner Shield Top Distance  
Active Face-to-Entrance Window Distance



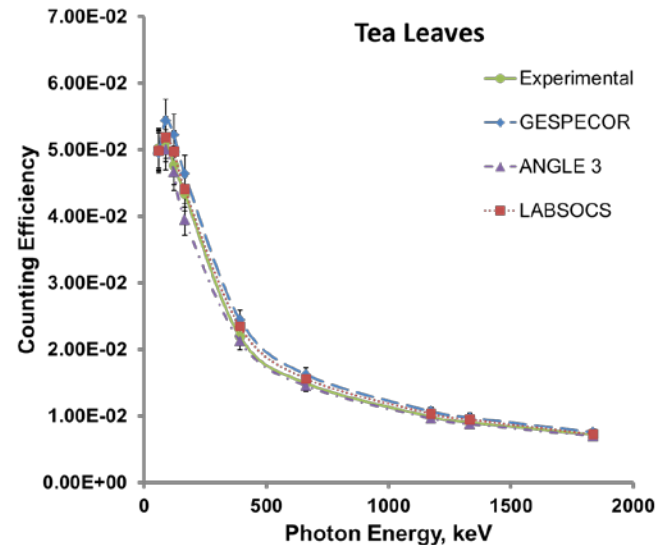
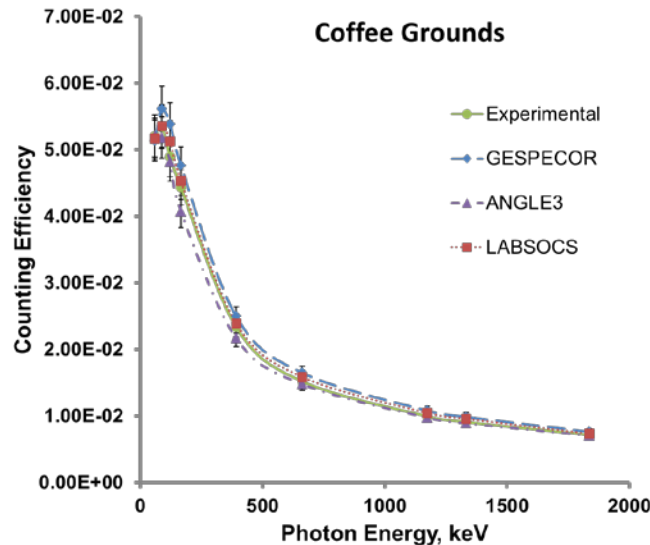
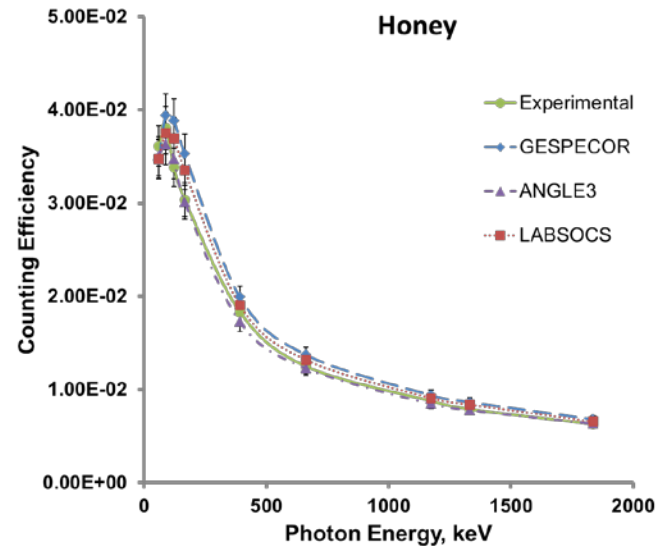
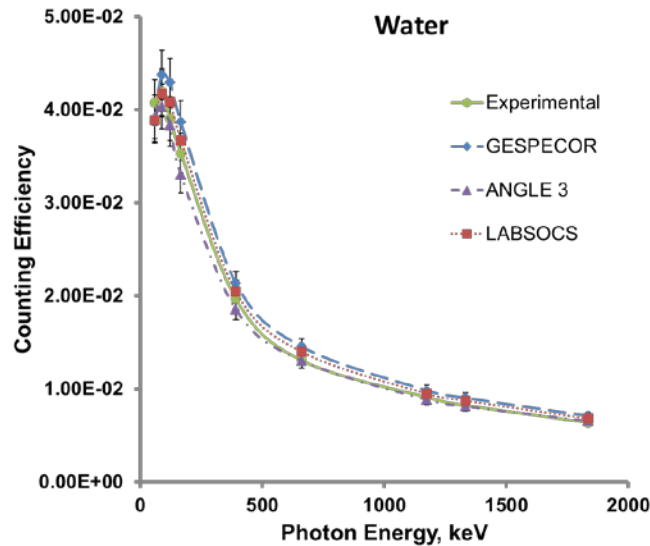
# The Effect of Food Density on Efficiency



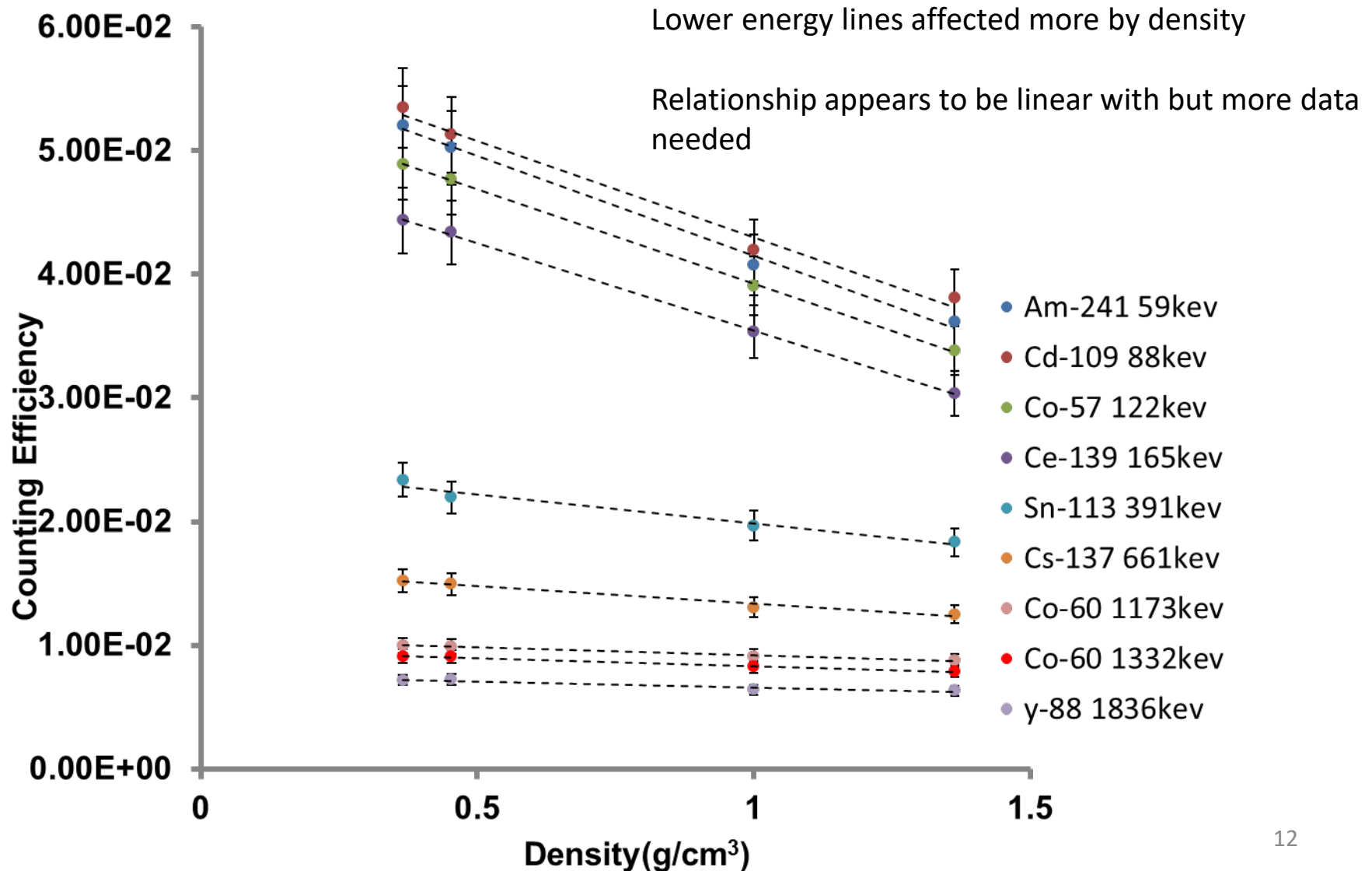
# Results- Sample Density



Effective at reproducing experimental results and limit cost



# Comparison of Measured Efficiencies with Change in Density



# Comparison of Measured Efficiencies with Change in Density

Food Product	Density	Elemental Analysis
Coffee	0.365	<b>C% 57.00 H% 7.00 O% 35.00 N% 1.00</b>
Tea	0.453	<b>C% 55.59 H% 6.10 O% 35.16 N% 3.02</b> <b>S% 0.07 P% 0.07</b>
Water	1	<b>H% 11.19 O% 88.81</b>
Honey	1.363	<b>C% 40.00 H% 7.00 O% 53.00</b>

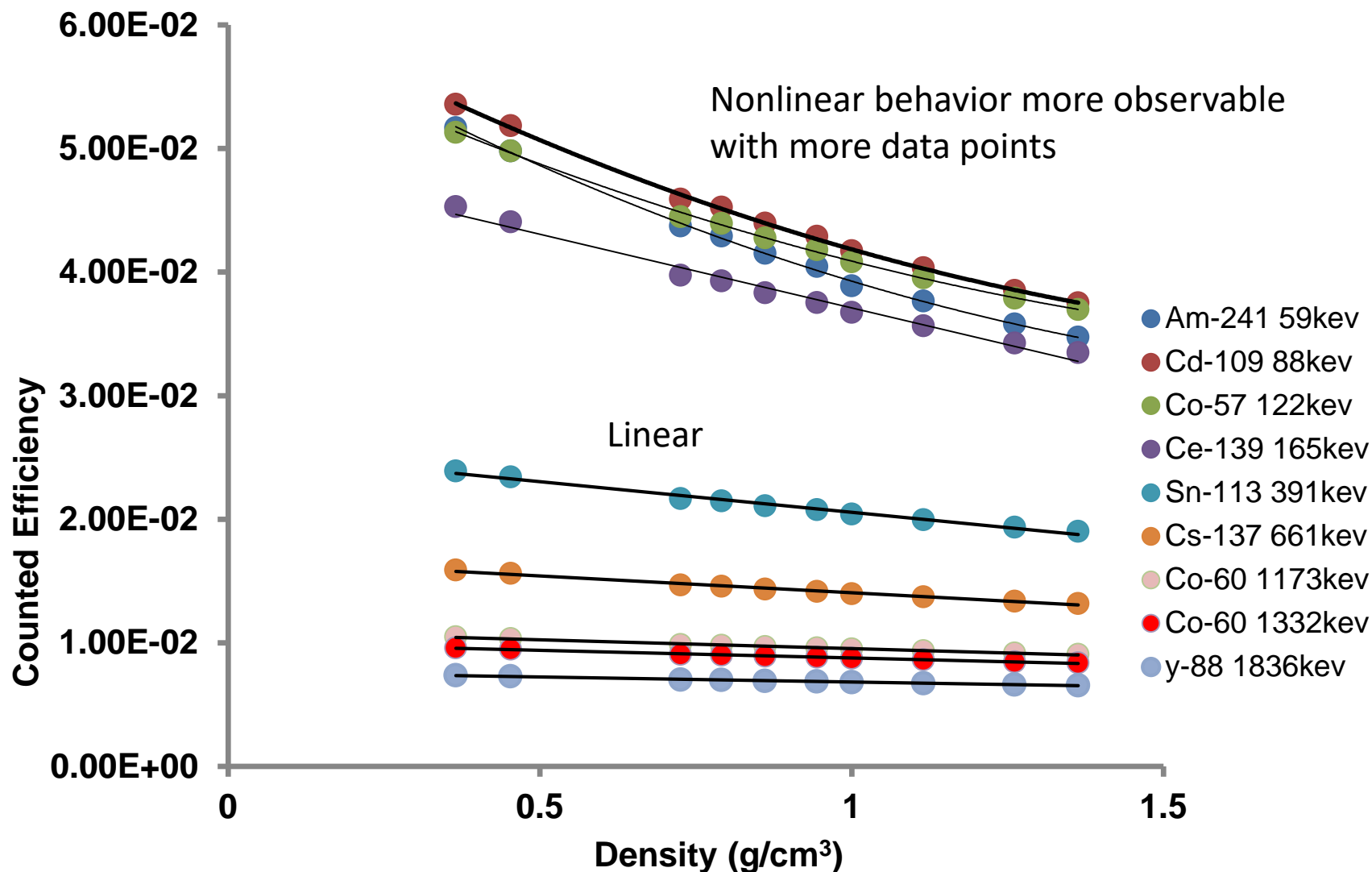
Organic Solvent	Density	Elemental Analysis
Triethylamine	0.7255	<b>C% 71.22 H% 14.94 N% 13.84</b>
Acetone	0.791	<b>C% 62.04 H% 10.41 O% 27.55</b>
DMF	0.944	<b>C% 49.30 H% 9.65 O% 21.89 N%19.16</b>
Glyme	0.9637	<b>C% 53.31 H% 11.18 O% 35.50</b>
Ethylene Glycol	1.115	<b>C% 38.70 H% 9.74 O% 51.55</b>
Glycerol	1.261	<b>C% 39.13 H% 8.76 O% 52.12</b>

Principle components in food are C, H, O, N

Using organic solutions to mimic food components, while improving range of densities that can be studied

Simulate radioactive solutions to minimize generating waste.

# Comparison of Computed Efficiencies with Change in Density



# Results of Food Spiked With $^{137}\text{Cs}$ and $^{60}\text{Co}$

**Spiked Activity:** The amount of radioactive material added to the food sample.

**Measured Activity:** The activity determined by measurement of the sample using a water based efficiency

**Corrected Activity:** The activity determined in sample using computed efficiencies with density corrections

Food Product	Density	Isotope	Spiked Activity (Bq/g)	Measured Activity (Bq/g)	%error	Corrected Activity (Bq/g)	%error
Water	1	$^{137}\text{Cs}$	0.64	0.66	3.29	0.62	2.93
		$^{60}\text{Co}$	1.03	0.96	6.98	0.98	5.41
			1.04	0.95	8.41	0.98	6.00
Coffee	0.365	$^{137}\text{Cs}$	4.09	4.76	16.45	3.94	3.55
		$^{60}\text{Co}$	6.61	6.87	3.99	6.32	4.33
			6.61	6.70	1.34	6.72	1.60
Tea	0.453	$^{137}\text{Cs}$	3.33	3.83	15.10	3.21	3.63
		$^{60}\text{Co}$	5.38	5.51	2.46	5.16	4.08
			5.38	5.41	0.65	5.08	5.47
Unknown Syrup	1.15	$^{137}\text{Cs}$	0.18	0.16	10.68	0.19	1.68
		$^{60}\text{Co}$	0.37	0.35	5.26	0.35	7.01
Honey	1.363	$^{137}\text{Cs}$	0.48	0.47	1.67	0.47	1.91
		$^{60}\text{Co}$	0.77	0.70	10.08	0.74	4.20
			0.78	0.70	9.42	0.74	3.94

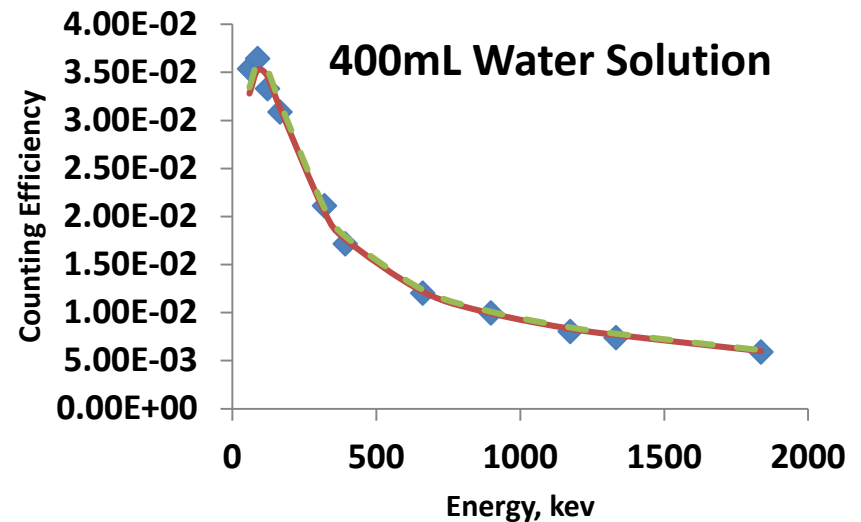
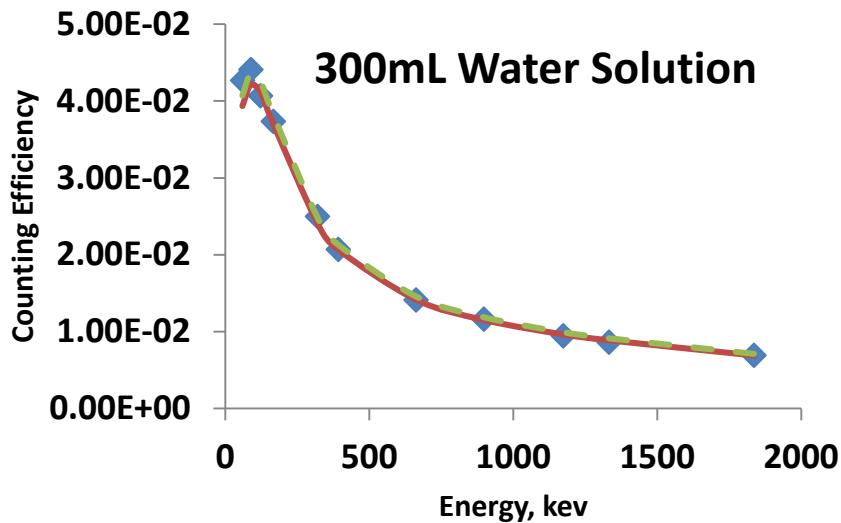
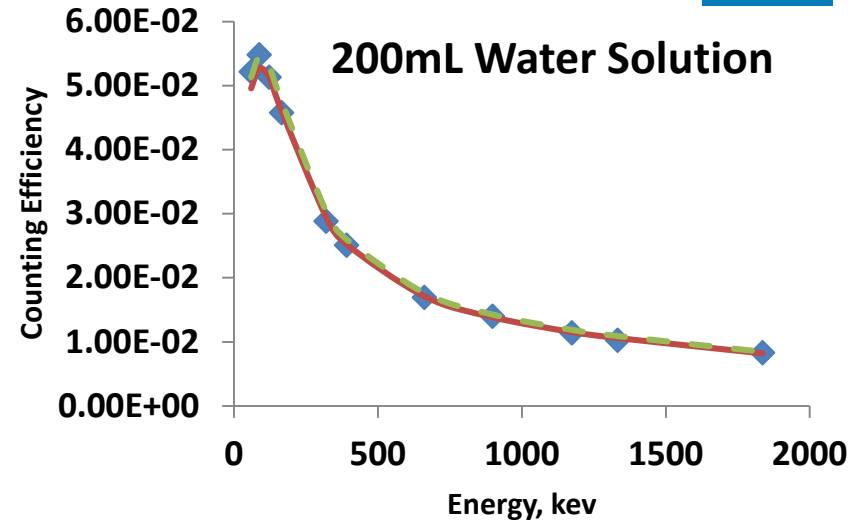
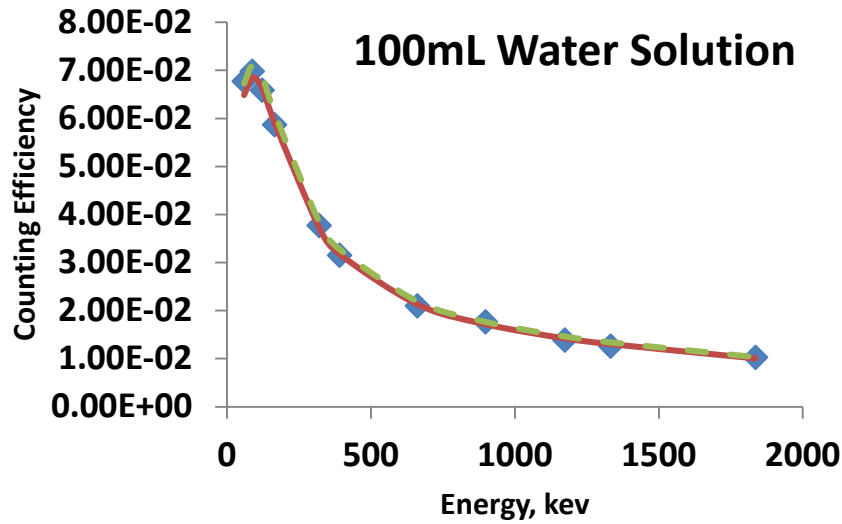


# Summing Corrections for $^{133}\text{Ba}$ and $^{134}\text{Cs}$ in Water

Nuclide	Known Activity	Uncorrected Activity (Bq/g)	% Diff.	LABSOCS Activity (Bq/g)	%Diff.	Gespecor Activity (Bq/g)	%Diff.	Angle Activity (Bq/g)	%Diff.
$^{133}\text{Ba}$	321.3	270	16.0%	330	2.7%	303	5.7%	300	6.6%
$^{134}\text{Cs}$	232.2	204	12.2%	225	3.1%	220	5.3%	223	4.0%

Our regulatory programs require a measurement accuracy to be within  $\pm 10\%$ . Efficiency computations with a 2% uncertainty assigned to each of the detector and sample parameters showed that a combined effect on counting efficiencies from all uncertainty components is on the order of 6%.

# Results – Sample Height



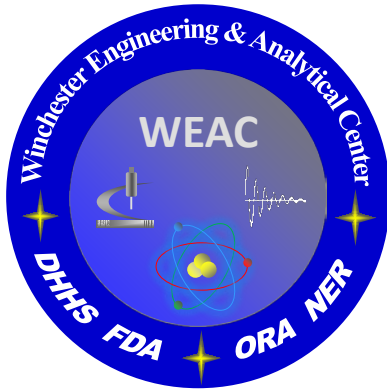
# Fission Products of <sup>235</sup>U in 400mL of Water

Radionuclide	Known, Bq/kg	±2s	Experimental Bq/kg	Diff, %	Labsocs Activity Bq/kg	Diff, %	Angle 3 Bq/kg	Diff, %	Gespecor	Diff, %
Ba-140	2.43E+04	7.54E+02	2.47E+04	1.55%	2.44E+04	0.31%	2.36E+04	2.98%	2.18E+04	10.38%
Ce-141	1.91E+04	5.33E+02	1.94E+04	1.83%	1.88E+04	1.32%	1.81E+04	4.99%	1.59E+04	16.54%
Ce-144	2.88E+03	1.27E+02	3.13E+03	8.56%	3.03E+03	5.10%	2.98E+03	3.36%	2.56E+03	11.21%
Cs-137	9.14E+01	3.75E+00	9.47E+01	3.59%	9.42E+01	3.04%	9.16E+01	0.20%	8.45E+01	7.57%
I-132	1.82E+03	8.18E+01	1.81E+03	0.39%	1.84E+03	1.26%	1.72E+03	5.35%	1.69E+03	7.00%
La-140	2.77E+04	7.48E+02	2.79E+04	0.68%	2.76E+04	0.40%	2.70E+04	2.57%	2.47E+04	10.87%
Mo-99	1.33E+03	4.14E+01	1.32E+03	1.11%	1.31E+03	1.86%	1.27E+03	4.86%	1.22E+03	8.60%
Nb-95	5.17E+03	1.55E+02	5.16E+03	0.29%	5.11E+03	1.25%	4.99E+03	3.57%	4.59E+03	11.30%
Nd-147	8.34E+03	4.33E+02	7.96E+03	4.51%	7.97E+03	4.40%	7.53E+03	9.67%	6.69E+03	19.75%
Ru-103	8.50E+03	2.72E+02	8.29E+03	2.49%	8.23E+03	3.20%	7.99E+03	6.02%	7.38E+03	13.20%
Tc-99m	1.40E+03	4.91E+01	1.49E+03	6.22%	1.45E+03	3.37%	1.39E+03	0.91%	1.22E+03	13.03%
Te-132	1.85E+03	6.47E+01	1.73E+03	6.42%	1.79E+03	3.18%	1.76E+03	4.80%	1.54E+03	16.70%
Zr-95	1.29E+04	4.53E+02	1.31E+04	1.26%	1.30E+04	0.49%	1.27E+04	1.83%	1.17E+04	9.56%

# Conclusions

- Computational Methods effective at predicting efficiencies over a range of densities
- Software packages are capable of delivering high throughput, accurate results in an emergency scenario
- Simulating radioactive solutions can help minimize generating waste saving cost of preparing samples and improve response time in the case of an emergency.

# Acknowledgements

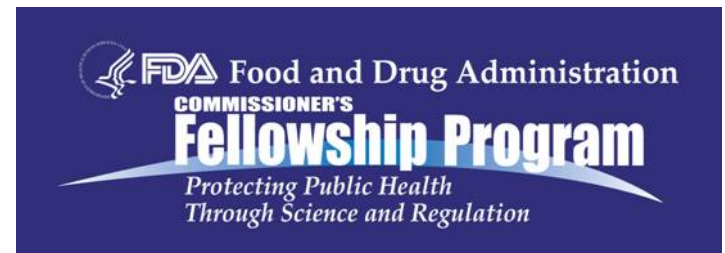


## **Winchester Engineering & Analytical Center (WEAC)**

Director: Brian Baker

Preceptor: Stephanie Healey

Dr. Zhichao Lin



## **Commissioner's Fellow Program**

Jeffery Rexrode

Susan Gantz

Kyle Boyd

